

D.M. JAMIL Vice President

Duke Power Catawba Nuclear Station 4800 Concord Road / CN01VP York, SC 29745-9635

803 831 4251 803 831 3221 fax

June 14, 2005

U.S. Nuclear Regulatory Commission

Washington, DC 20555-0001

ATTENTION: Document Control Desk

Subject:

Duke Energy Corporation

Catawba Nuclear Station Unit 1

Docket No. 50-413

Response to Revised NRC Order (EA-03-009) which Established Interim Inspection Requirements for Reactor Pressure Vessel Heads at Pressurized Water

Reactors

Revised NRC Order (EA-03-009) which established interim inspection requirements for reactor pressure vessel heads at Pressurized Water Reactors (PWR) required a bare metal visual (BMV) examination of 100 percent of the reactor pressure vessel (RPV) head surface (including 360° around each RPV head penetration nozzle). Additionally, revised NRC Order (EA-03-009) required that a report detailing the inspection results be submitted within 60 days of returning the plant to operation. Participating inspectors must be ASME XI, VT-2 qualified.

During the Catawba Unit 1 Refueling Outage (1EOC15), inspections described in response to IV.C.(3), Pressurized Water Stress Corrosion Cracking (PWSCC) Susceptibility Category (Low), of the revised order were completed. With respect to NRC Order EA-03-009, Catawba Unit 1 is considered a PWSCC low susceptibility plant based on reactor vessel head temperature and operational history. The inspection conformed to the requirements of EPRI Report 1007842, Visual Examination for Leakage of PWR Reactor Head Penetrations, Revision 2, and Order EA-03-009.



A101

There were no concerns with the structural integrity of the reactor vessel head. The head is in very good condition with little oxidation. Localized areas were observed with minor surface discolorations; however, these have no effect on the structural integrity. Evidence of leakage was solely in the form of oxidation trails and was clearly associated with a historical leak path from above. These same trails were observed in the 1EOC13 inspection. In no case was a concentrated residue or deposit observed. There was no indication or evidence of leakage through the Alloy 600, J-groove weld of the Control Rod Drive Mechanisms or other Alloy 600 locations. The pressure retaining function of the vessel has not been degraded.

These BMV inspections were performed by VT-2 qualified inspectors

There are no NRC commitments contained in this letter.

Inquiries on this matter should be directed to A. Jones-Young at (803) 831-3051.

Very truly yours,

D.M. Jamil

Site Vice President

U.S. NRC June 14, 2005 Page 3

D. M. Jamil, being duly sworn, affirms that he is the person who subscribed his name to the foregoing statement, and that all matters and facts set forth herein are true and correct to the best of his knowledge.

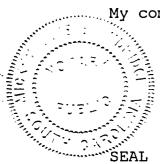
D. M. Jamil, Site Vice President

Subscribed and sworn to me: 6-14-05

\_\_\_, Notary Public

My commission expires:

7-10-2012



U.S. NRC June 14, 2005 Page 4 XC:

W. D. Travers
U. S. Nuclear Regulatory Commission
Regional Administrator, Region II
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, GA 30303

S. E. Peters NRC Project Manager U. S. Nuclear Regulatory Commission Mail Stop O-8 H12 Washington, DC 20555-0001

E. F. Guthrie Senior Resident Inspector U. S. Nuclear Regulatory Commission Catawba Nuclear Site

H. J. Porter Division of Radioactive Waste Management South Carolina Bureau of Land and Waste Management 2600 Bull Street Columbia, SC 29201